UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 2, 1979

MEMORANDUM FOR: H. R. Denton, Director, NRR
R. H. Vollmer, Director, TMI-2 Support

FROM: J. T. Collins, Deputy Director, TMI-2 Support

SUBJECT: TMI-2 WEEKLY STATUS REPORT

Plant Status:

Natural circulation operation continues with the following core and plant conditions:

Pressurizer Temp.	- 398°F	Rx Building	- 0.7 psig (negative)
Pressure	- 326 psig		
T_{hot}	- 166°F	RC Leak Rate	- 0.5 gpm
T_{cold}	- 158°F	Rx Building Water Level	- 289.5 ft
Max Tc	- 270°F		

The temperature in the pressurizer has been coming down at about 2°F/day. Based on this we expect that in about six days the temperature will be down to the set point of 385°F. During the week we have been closing down on the by-pass valve to maintain the temperature in the "A" OTSG at about 160°F. At the present time the valve is about 16% open and holding. Weekly motor megger readings and daily switchbox megger readings for DH-V-1, DH-V-171, and DH-V-2 remained the same. The switchbox for CF-V-115 showed a new short circuit, however, the valve is open. GPU is developing a recommended course of action should the water level reach 290.5'. This should be available early this week.

A containment air sample taken on 06/26/79 showed the following results.

Kr-85	1.0 uCi/cc
Xe-131m	8.6×10^{-2} uCi/cc
Xe-133	1.5×10^{-2} uCi/cc
I-131	1.2×10^{-4} uCi/cc

Co-58, Co-60, Cs-134, and Cs-137 were all less than MDA. Comparing the results of this sample with the sample taken on 05/31/79, it appears that the iodine is decaying with an eight-day half-life.

The "A" waste gas compressor is still out of service. Repairs should be completed by 07/01/79 at which time the "B" compressor will be removed and repaired.

On June 27, we received a report from Met Ed which summarizes the present status of efforts to determine the radiation dose rates inside containment from the recorded dome monitor (HP-R-214) and to quantitatively determine the amount of fission product activity released into the U-2 Rx Building. Copies of this report are being made available to interested people within NRC.

The primary coolant sample taken on 06/27/79 continued to show a high chloride level, 4.8±1 ppm. This is essentially the same as the sample taken on 06/19/79 which showed a chloride level of 6.0 ppm. The chemistry group is looking into a possible source of chloride. One principal source is the boric acid which showed a level of between 2-3 ppm.

We still have not received the analytical results of the radiation measurements taken on the equipment hatch or the results of the gamma probe readings through the Rx Building 605 penetration. Both reports are due 07/02/79. With regard to the latter the penetration extends into the Rx Building about 10" and is about 2' over the water level. Preliminary results indicate a radiation level of 220 R/hr over the sump. This is based on the non-directional probe reading of 30 R/hr.

Modifications:

Interim Solid Waste Storage Facility - Culverts have been sealed with epoxy and the stripable coating should be applied this week. Three shield blocks have been delivered and the rest will be shipped this week. Operator training continued during the week. Provisions for flood protection as recommended by NRR have not yet been instituted. Estimated completion date for the interim facility is 07/06/79. Design criteria for the long term storage facility was approved on 06/28/79. No construction schedule for this facility has been established.

Tank Farm - The steam eductors for the lower tanks were successfully tested and water was transferred to the upper tanks. The steam eductors for the upper tanks are being modified and should be tested this week. Based on comments received from IE additional shielding is being added to the transfer line from the Rx Building spray pump to the tank farm. The tank farm should be available to accept waste by 07/06/79.

"B" OTSG Long Term Cooling - Provisions for venting the steam generator back to the condenser when filling the generator have been made. Most procedures for operation of the system should be completed by 07/02/79.

EPICOR-II - All work except the shielded transfer bell should be completed by 07/15/79. The transfer bell is scheduled to be delivered by 07/23/79.

Sample Sink-U-2 - All work was stopped on this because the licensee did not have design criteria acceptable to the staff. We expect to see revised criteria by 07/03/79.

Pressure Volume/Control System - All procedures necessary to separate this system should be completed by 07/06/79. Instead of a separate control room as previously planned all major controls will be routed to the U-2 control room. No firm date for completion of this work has been established.

Alternate Decay Heat Removal - System tie-in still scheduled for 07/15/79. During the past week it was found that the pipe through the penetration was commercial grade instead of nuclear grade. Westinghouse is redoing their stress analysis. The pipe has been radiographed and found acceptable. IE Construction Inspectors are following this. We will request that Westinghouse submit their stress analysis for review by NRR.

Environmental:

There have been no significant releases to the environment in the past week. Iodine-131 levels at the discharge point remained constant at about 10^{-13} to 10^{-14} uCi/cc. All environmental surveys and samples showed nothing above background.

At the request of Commissioner Gilinsky I attended a town meeting scheduled by Congressman Allen Ertel at Middletown on Friday evening, June 29. Approximately 500 local residents attended. Most of the questions directed to Commissioner Gilinsky centered around the proposed start-up of Unit 1. Very few questions were directed to the ongoing recovery program at Unit 2.

J. T. Collins

J. T. Collins, Deputy Director
TMI-2 Support

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